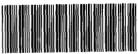
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ROCKY FLATS PLANT

1E687



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MONTHLY
ENVIRONMENTAL
MONITORING
REPORT

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MARCH 1989 ENVIRONMENTAL MONITORING REPORT ROCKY FLATS PLANT

This report summarizes the effluent and environmental monitoring programs at the Rocky Flats Plant for the month of March 1989

Included in the report are monitoring results for radioactive and nonradioactive airborne effluents continuously sampled from Plant buildings, Tables I and II Tables III through V summarize environmental monitoring data from the Rocky Flats Plant ambient air sampling network. This network is comprised of continuously operating air samplers located on plantsite, around the Plant boundary, and in neighboring communities.

Water sampling results for radioactive constituents are given in Tables VI through VIII Results are summarized for Plant surface water control ponds, for nearby drinking water reservoirs, and for tap water for neighboring communities. Nitrate monitoring for Great Western Reservoir and Standley Lake, the two drinking water reservoirs which can receive surface water discharges from the Plant, are summarized in Table IX

The Environmental Protection Agency (EPA) has issued to the Plant a National Pollutant Discharge Elimination System (NPDES) permit for control of surface water discharges. Water sampling results associated with the NPDES permit, as well as applicable discharge limitations imposed by that permit, are reported in Table X Analytical results for nonradioactive parameters in water at the Walnut Creek at Indiana Street location are summarized in Table XI Daily flow data for surface water from the two Plant drainage systems are given in Tables XI, XII, and XIII

The Rocky Flats Plant Environmental Monitoring Program includes evaluating plant compliance with all relevant guides, limits, and standards. All average results of monitoring effluent and ambient samples complied with the applicable standards as specified in Executive Order 12088 (rules, regulations, and requirements of the Department of Energy)

The data provided in this report are provided as a matter of comity and should not be construed as an application for a permit or license, or in support of such an application. Approval of the Department of Energy should be obtained prior to publication of any data contained within this report

Table I 1989 Plutonium and Uranium Airborne Effluent Data

	Plutonium		Uranjum	
	(01/19/89 Release	<u>- 02/20/89 - Feb)</u> CMax	<u>(01/20/89</u> Release	- 02/21/89 - Feb) CMax
Month	(uC1)	(pC1/m3)	(uC1)	(pC1/m3)
CY 1988	15 33	0 023 ± 0 0052	11 93	0 009 ± 0 0009
January	0 33	0 005 ± 0 0005	0 15	0 000 ± 0 0001
February	0 15	0 001 ± 0 0001	0 20	0 001 ± 0 0002
March	0 07*	0 001 ± 0 0001	0 04	0 002 ± 0 0002
April				
May				
June				
July				
August				
September				
October				
November				
December				
Year to Date	0 55*	0 005 ± 0 0005*	0.39	0 001 ± 0 0002

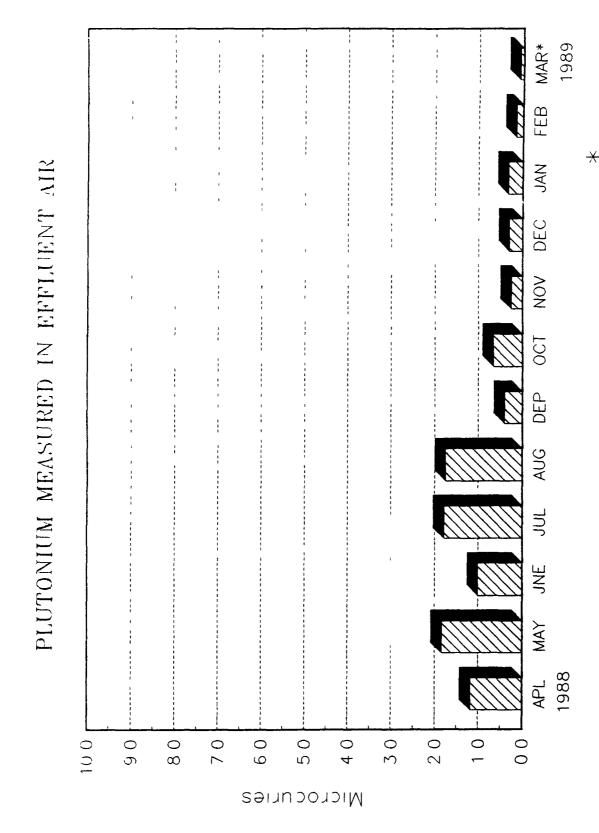
^{* 2} incomplete analyses

NOTE The plutonium, uranium, americium, and beryllium measured concentrations in this report include values that are less than the corresponding calculated minimum detectable concentrations (MDC's). In some cases, the values are less than zero. This method of reporting began in January 1981. These negative values result when the measured value for the laboratory reagent blank is subtracted from an analytical result which was measured as a smaller value than the reagent blank. This may happen when measuring concentrations which are very close to zero.

Table II 1989 Tritium and Beryllium Airborne Effluent Data

	Tr	ıtıum	<u>Beryllıum</u>	
	(01/20/89 -		(01/19/89 -	<u> </u>
Month	Release (C1)	CMax (pC1/m3)	Release	CMax (ug/m3)
Honen		(pc1/m3)	(grams)	(ug/iiis/
CY 1988	0 015	417 ± 250	0 1383	0 00041
January	0 001	97 ± 145	0 0285	0 00033
February	0 002	166 ± 120	-0 0392	-0 00005
March	0 007	389 ± 220	-0 0025	0 00000
April				
May				
June				
July				
August				
September				
October				
November				
December				
Year to Date	0 010	389 ± 220	-0.0132	0.00022
rear to pate	0 010	309 I 220	-0.0132	0 00033

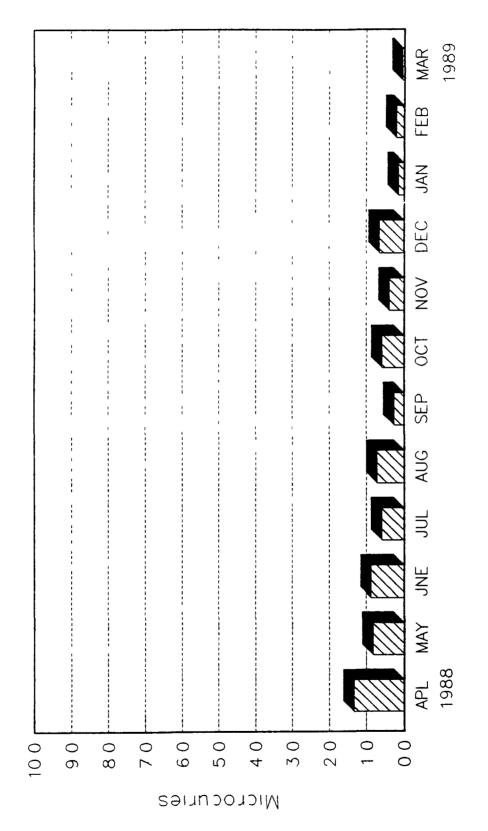
NOTE Beryllium measured at 36 other screening locations was below the screening level of 0 $1\,\mathrm{gram}$ per month



Analysis in Progress for 2 Samples

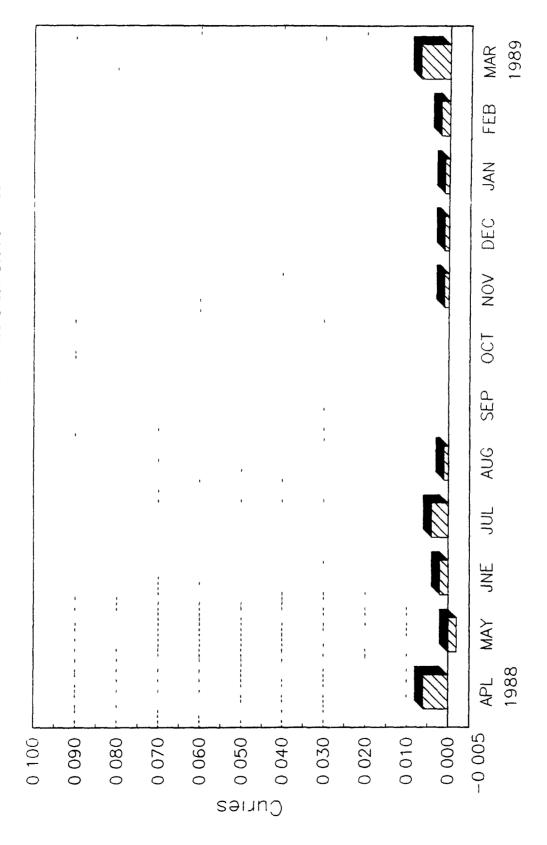
Incomplete analysis,

- Page 3 -

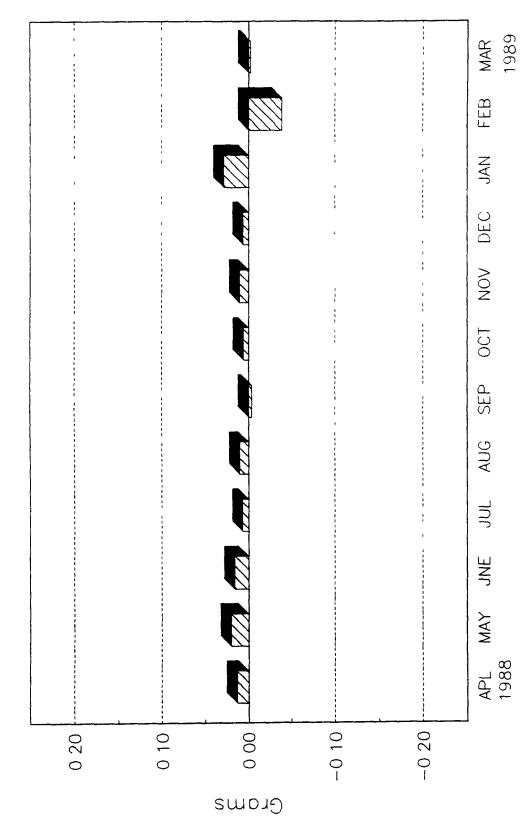


- Page 4 -

TRITICAL MEASURED IN EFFLUENT ALL



BERYLLIUM MEASURED IN EFFLUENT AIR



- Page 6 -

Table III Plutonium at Selected Onsite Ambient Air Locations (02/21/89 - 03/21/89)

			<u>Concentration</u>		
ation	<u>n</u>	Volume (m3)	Point <u>Estimate</u>	± Error	
05 06	2 2	28000 *	0 000117	0 000013	
07	2	23000	0.000186	0.000023	
08	2	32000	0.000336	0 000035	
09	2	31000	0.000316	0.000033	

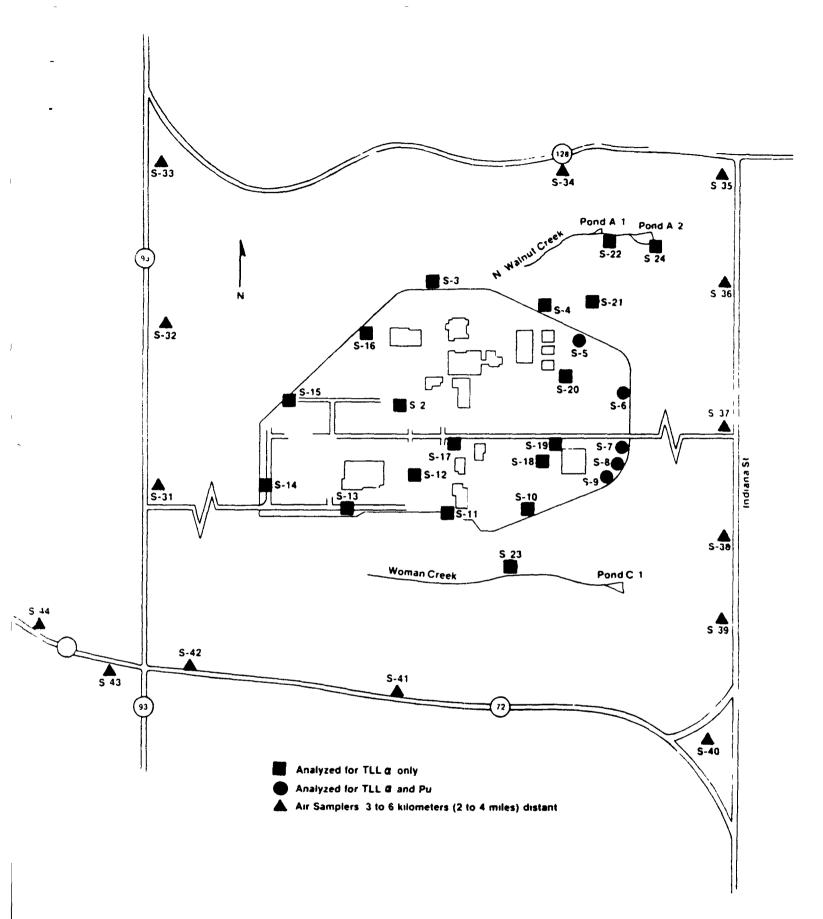
ncomplete Analyses

Total long-lived alpha at the remaining 18 onsite ambient air samplers was below the screening level of 0.01 pCi/m³

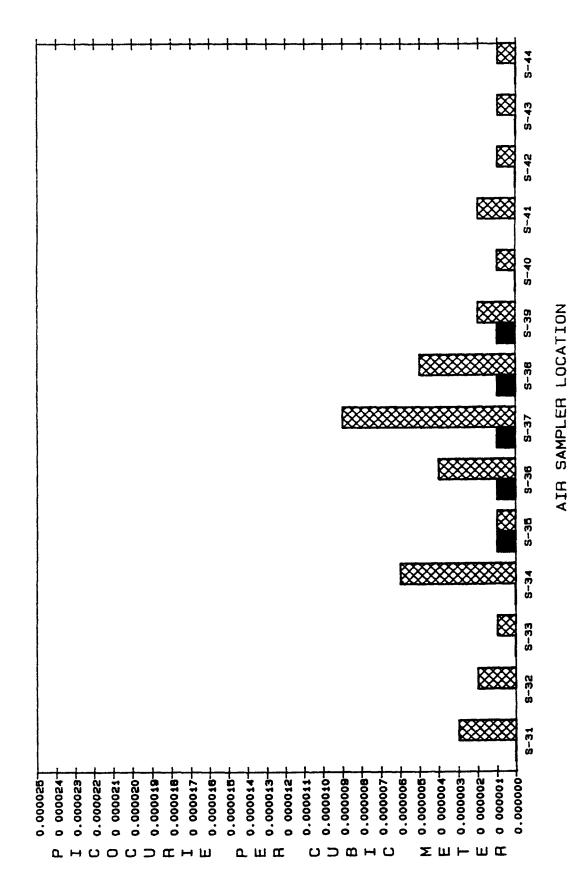
Table IV. Plutonium in Perimeter Ambient Air (02/28/89 - 03/28/89)

			<u>Concentrati</u>	on (pC1/m3)
<u>Location</u>	<u>n</u>	Volume (m3)	Point <u>Estimate</u>	± Error
S-31	1	19000	0 000000	0 000002
S-33	ī	29000	0.000000	0 000001
S-34	ī	30000	0 000000	0 000001
S-35	ī	32000	0 000001	0 000001
S-36	ì	30000	0 000001	0 000001
S-37	1	33000	0 000001	0 000001
S-38	1	28000	0 000001	0 000001
S-39	1	31000	0 000001	0 000001
S-40	1	30000	0 000000	0 000001
S-42	1	26000	0 000000	0 000001
S-43	1	31000	0 000000	0 000001
S-44	1	29000	0 000000	0 000001

Note Ambient air samplers S-32 and S-41 were inoperational during this period



Location of Onsite and Plant Perimeter Ambient Air Samplers (Portions of figure are not to scale)



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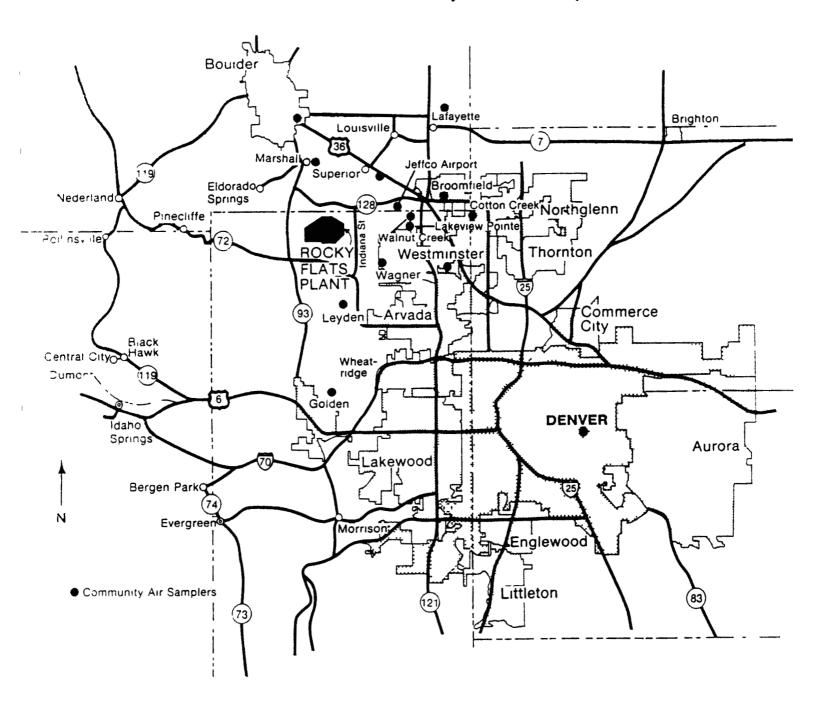
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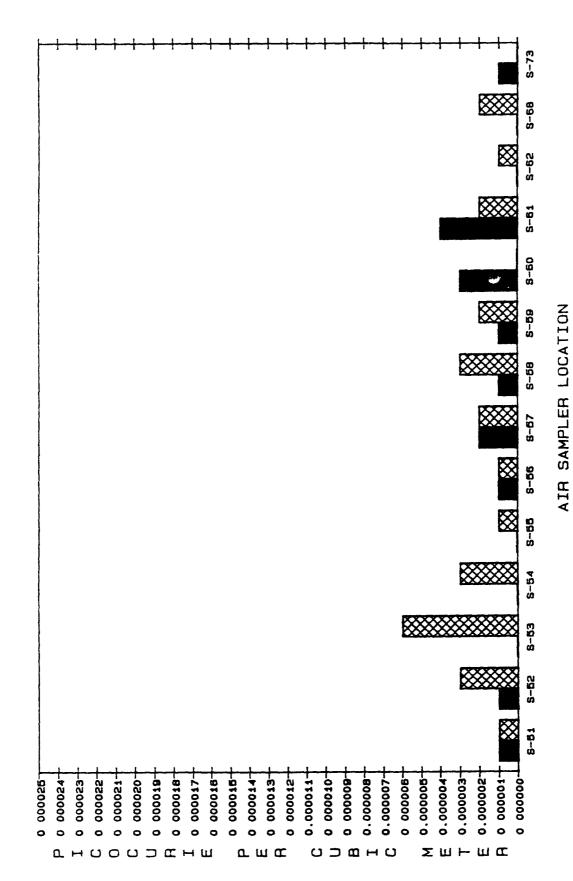
- Page 10 -

Table V Plutonium in Community Ambient Air (03/01/89 - 03/29/89)

			Concentr	ation (pCi/m3)	
Location	Community Name	<u>n</u>	Volume (m3)	Point <u>Estimate</u>	± Error
S-51	Marshall	1	27000	0 000001	0 000001
S-52	Jeffco Airport	1	34000	0 000001	0 000001
S-53	Superior	1	29000	0 000000	0 000001
S-54	Boulder	1	29000	0 000000	0 000001
S- 55	Lafayette	1	29000	0 000000	0 000001
S-56	Broomfield	1	25000	0 000001	0 000001
S- 5 7	Walnut Creek	1	30000	0 000002	0 000001
S-58	Wagner	1	29000	0 000001	0 000001
S- 59	Leyden	1	32000	0 000001	0 000001
S-60	Westminster	1	21000	0 000003	0 000002
S-61	Denver	1	9000	0 000004	0 000004
S-62	Golden	Ĩ	37000	0 000000	0 000001
S-68	Lakeview Pointe	ī	37000	0 000000	0 000001
S-73	Cotton Creek	ī	27000	0 000001	0 000001

Location of Community Ambient Air Samplers





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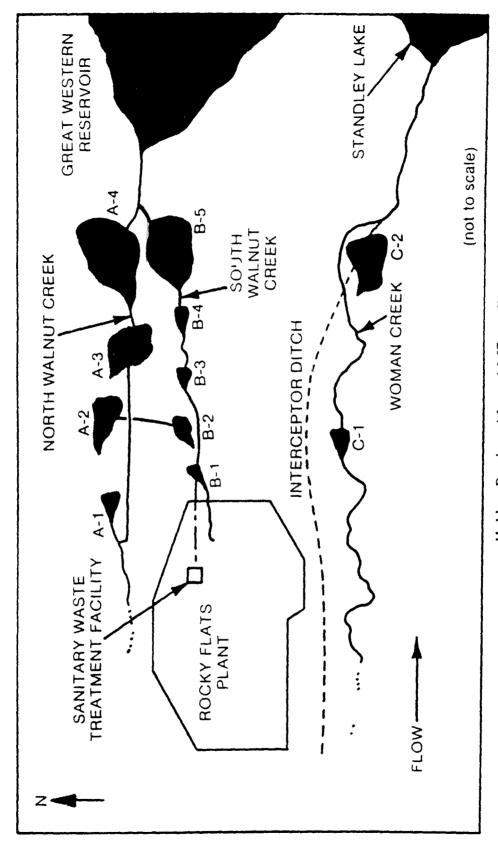
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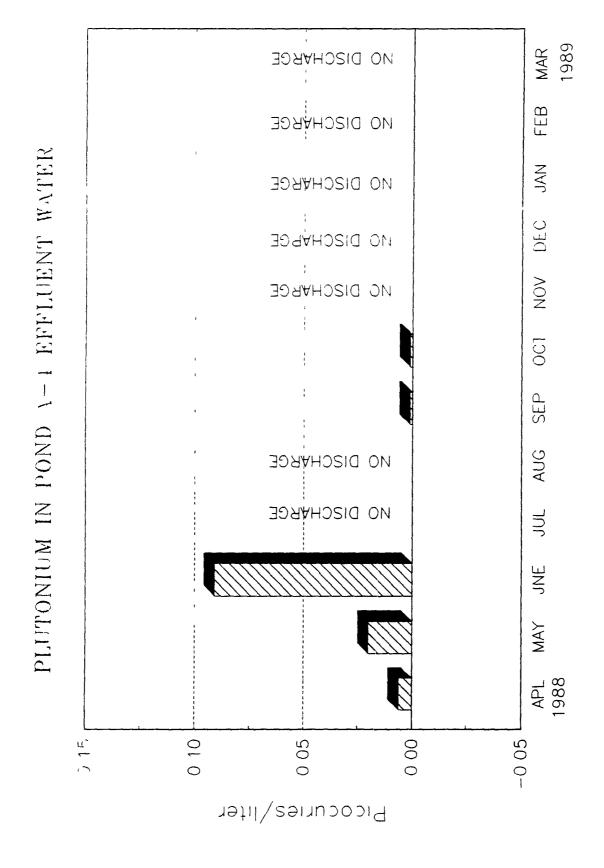
Table VI Onsite Water Sample Results - Plutonium, Uranium, and Americium

Holding Pond Outfall (po	21/1)		
<u>Location</u>	<u> Plutonium</u>	Uranium	<u>Americium</u>
Pond A-4			
No Discharge			
Average Concentration			
Pond B-5			
No Discharge			
Average Concentration			
Pond C-1			
03/06/89 - 03/10/89 03/13/89 - 03/17/89 03/20/89 - 03/24/89 03/27/89 - 03/31/89	-0 001 ± 0 005 0 001 ± 0 007 * 0 035 ± 0 008	0 29 ± 0 16 0 46 ± 0 16 0 68 ± 0 17 0 34 ± 0 165	0 006 ± 0 006 0 005 ± 0 006 -0 002 ± 0 008
Average Concentration	*	0 45 ± 0 17	*
Pond C-2 No Discharge			
Average Concentration			
Walnut Creek at Indiana			
03/06/89 - 03/10/89 03/13/89 - 03/17/89	0 002 ± 0 009 0 004 ± 0 017	4 92 ± 0 37 5 65 ± 0 34	0 025 ± 0 011 0 003 ± 0 018
Average Concentration	0 003 ± 0.014	5 29 ± 0.35	0.014 ± 0.014

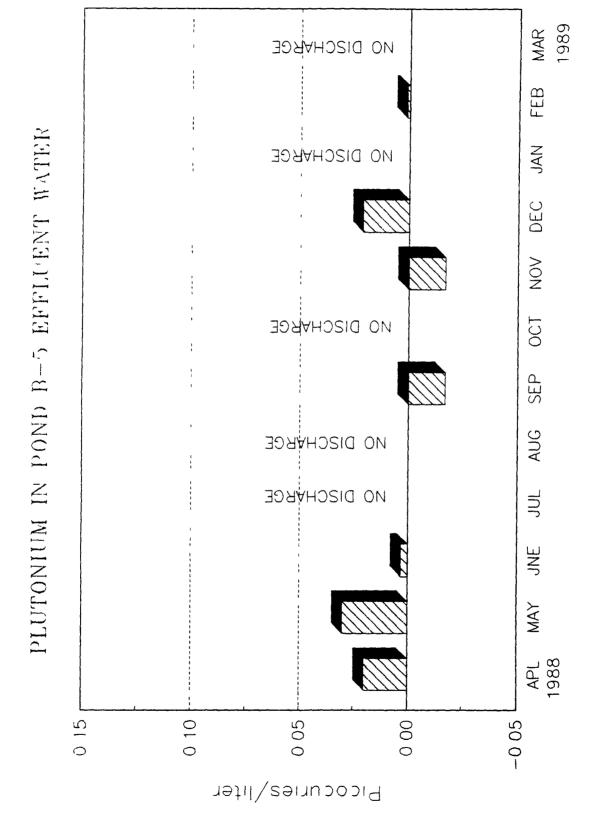
* Analyses Incomplete



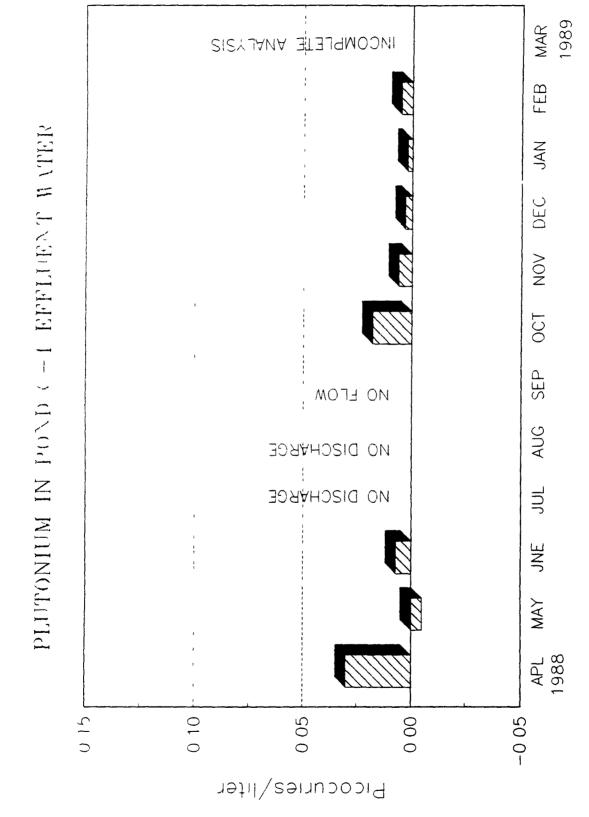
Holding Ponds and Liquid I filuent Watercourses



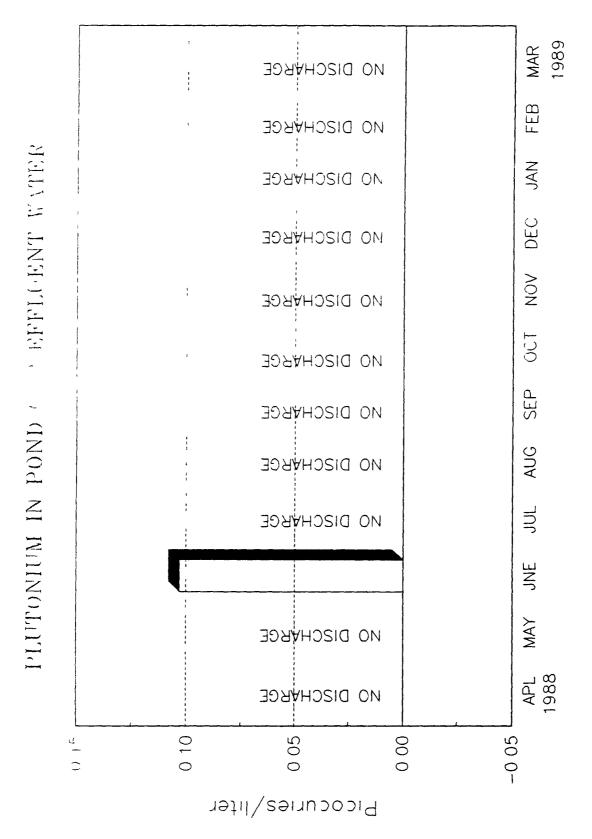
- Page 16 -



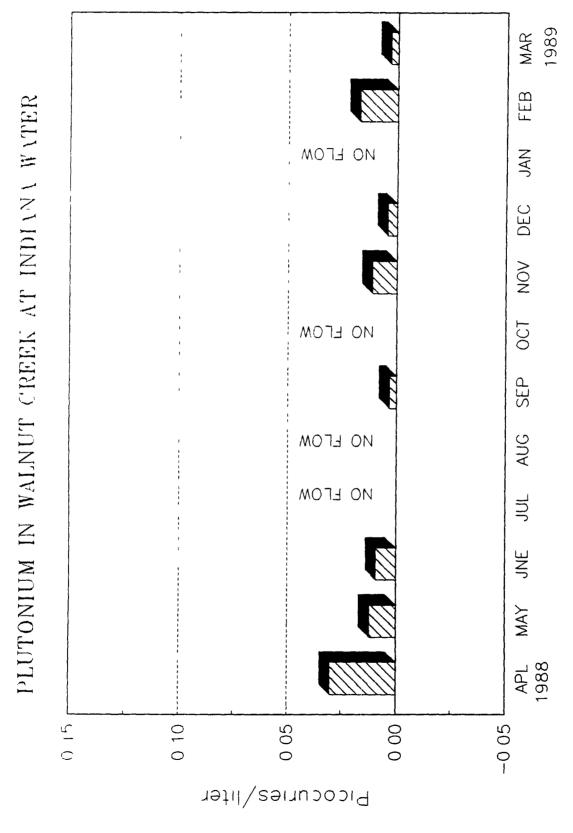
- Page 17 -



- Page 18 -



- Page 19 -



- Page 20 -

Table VII. Offsite Water Sample Results - Plutonium, Uranium, and Americium

Reservoirs ((pCi	/11	١
110001 1011 0		, , ,	,

<u>Location</u>	<u>n</u>	Plutonium	Uranium	Americium
Great Western	1*	**	1 66 ± 0.20	**
Standley Lake	1*	0.000 ± 0.007	1 66 ± 0 21	**

Community Tap Water (pC1/1)

<u>Location</u>	<u>n</u>	Plutonium	Uranium	<u>Americium</u>
Boulder	1*	-0 002 ± 0 007	0 55 ± 0 18	**
Broomfield	1*	**	1.36 ± 0.19	**
Westminster	1*	$0\ 001\ \pm\ 0\ 007$	061 ± 017	**

^{*} Plutonium, uranium and americium analyses were performed on one sample composited from four weekly grab samples.

^{**} Analyses Incomplete

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Table VIII Onsite and Offsite Water Sample Results - Tritium

<pre>Tritium (pCi/l)</pre>				
<u>Location</u>	<u>n</u>	C _{M1n1mum}	C _{Max1mum}	C _{Average}
Pond A-4		No Discharge		
Pond B-5		No Discharge		
Pond C-1	3	-440 ± 316	51 ± 325	-188 ± 320
Pond C-2		No Discharge		
Walnut Creek at Indiana	5	-270 ± 288	88 ± 389	- 93 ± 318
Boulder	4	-109 ± 392	271 ± 329	96 ± 329
Broomfield	4	-150 ± 389	82 ± 324	- 15 ± 326
Great Western	4	- 31 ± 288	162 ± 330	62 ± 329
Standley	4	- 27 ± 393	243 ± 297	47 ± 329
Westminster	4	-104 ± 289	59 ± 327	- 20 ± 326

Table IX Offsite Water Sample Results - Nitrate as Nitrogen

Nitrate (as N) at Great Western Reservoir

Sample Date	Nitrate (as N) (mg/l)
03/02/89	<0 02
03/09/89	0 06
03/16/89	0 06
03/23/89	0 21
03/30/89	0 22

Nitrate (as N) at Standley Lake

<u>Sample Date</u>	Nitrate (as N) (mg/l)
00/00/00	2 22
03/02/89	0 22
03/09/89	0 24
03/16/89	0 06
03/23/89	0 07
03/30/89	0 05

NOTE For some nonradioactive parameters, the concentrations that are measured at or below the minimum detectable concentration (MDC) are assigned to MDC. The less than symbol (<) indicates MDC values and calculated values that include one or more MDC's

Table X NPDES Permit Water Sample Results

Discha	rqe	001	(Pond	B-3)
		charg		

Parameters Blochem Oxygen Demand, 5 Day Total Suspended Solids Nitrates as N Total Chromium Total Phosphorus Oil and Grease, Visual Total Residual Chlorine Fecal Coliforms #/	mg/] mg/1 mg/1 mg/1 mg/1	Measured 30-Day Average No Discharge	Limits 30-Day* Average 10 30 10 0 05 8 NA NA 200	Measured Daily <u>Maximum</u> No Dischar	Limits Daily Maximum ge 25 NA NA 0 1 NA NA NA 0 5
<u>Parameter</u> pH	s u	Measured Daily Minimum No Discharge	Limits Daily Minimum 6 0	•	Limits Daily Maximum ge 90
Discharge 002 (Pond A-3) No Discharge Parameters Nitrates as N	mg/l	Measured 30-Day Average No Discharge	Limits 30-Day* Average 10	•	<u>Limits</u> Daily <u>Maximum</u> rge 20
рН	s u	Measured Daily Minimum No Discharge	Limits Daily Minimum 6 0	Measured Daily Maximum No Discha	<u>Limits</u> Daily <u>Maximum</u> rge 9 0
Discharge 003 (RO Pilot Plant) No Discharge Parameter pH	s.u	Measured Daily Minimum No Discharge	<u>Limits</u> Daily <u>Minimum</u> 6.0		<u>Limits</u> Daily <u>Maximum</u> ge 90

^{*} This limitation applies when a minimum of 3 consecutive samples are taken during separate weeks

Table X NPDES Permit Water Sample Results (Continued)

Discharge 004 (RO Plant) No Discharge		Measured Limits Measured Limits
Parameters Total Suspended Solids Total Organic Compounds Total Phosphorus Nitrates as N Total Chromium Total Residual Chlorine	mg/l mg/l mg/l mg/l mg/l	30-Day 30-Day* Daily Average Average Maximum Maximum No Discharge 25 22 30 8 12 10 20 0 05 0 1 NA 0 5
Fecal Coliform	#/100 ml	7-Day 7-Day 30-Day 30-Day Average Average Average No Discharge 400 No Discharge 200
рН	s u	Daily Daily Daily Daily Minimum Maximum Maximum No Discharge 60 No Discharge 90
Discharge 005 (Pond A-4) No Discharge		
Parameters pH S U Nitrates as N mg/ Nonvolatile mg/ Suspended Solids	1	C <u>Mınımum</u> C <u>Maxımum</u> C <u>Average</u> No Dıscharge
<u>Discharge 006 (Pond B-5)</u> No Discharge		
Parameters pH S U Nitrates as N mg/ Nonvolatile mg/ Suspended Solids	1	C <u>Mınımum</u> C <u>Maxımum</u> C <u>Average</u> No Dıscharge
<u>Discharge 007 (Pond C-2)</u> No Discharge		
Parameters pH S U Nitrates as N mg/ Nonvolatile mg/	1	C _{Mınımum} C _{Maxımum} C _{Average} No Dıscharge

Nonvolatile Suspended Solids

MARCH 1989

Table XI Water Sample Results, Nonradioactive Parameters

Walnut Creek at Indiana Street

<u>Parameters</u>		<u>n</u>	C _{Mınımum}	C _{Maximum}	C _{Average}
рН	s u	3	6 5	6 8	N/A
Nitrates as N	mg/l	3	0 20	0 80	0 46

Total Volume (gallons) = 799,000

Table XII. Daily Flow Data Recorded at the Walnut Creek at Indiana Gaging Station Ponds A-4 and B-5, March, 1989

(Walnut Creek Drainage)

<u>DATE</u>	WALNUT CREEK AT INDIANA (gallons)	POND A-4 (gallons)	POND B-5 (gallons)
03/01/89	No Flow	No Discharge	No Discharge
03/02/89	11 H	11 H	11 11
03/03/89			
03/06/89	H H		
03/07/89	M 11	11 11	11 11
03/08/89	262,000	11 11	11
03/09/89	306,000	11 11	11 11
03/10/89	182,000	11 11	11
03/13/89	26,000	11 11	11 11
03/14/89	23,000	11 11	11 11
03/15/89	No Flow	11 11	11 11
03/16/89	11 11	11 11	11 11
03/17/89	u u	u u	tf (f
03/20/89	11 11	11 11	17 11
03/20/89	11 11	15 15	11
	n 11	# 11	11 11
03/22/89	n n	11 11	19 11
03/23/89	11 11	11 11	19 14
03/27/89	11 11		11 11
03/28/89	11 H	H 11	и и
03/29/89			
03/30/89	11 11	17 19	11 11
03/31/89	11 11	11 11	H II
TOTAL VOLUME	799,000*	No Discharge	No Discharge

^{*} Precipitation/Runoff

Table XIII Daily Flow Data Recorded at Ponds C-1 and C-2 During March, 1989

(Woman Creek Drainage)

<u>DATE</u>	POND C-I (gallons)	POND C-2 (gallons)
03/01/89	628,000	No Discharge
03/02/89	512,000	" "
03/03/89	572,000	16 75
03/06/89	1,282,000	11 11
03/07/89	384,000	11 11
03/08/89	556,000	11 11
03/09/89	512,000	11 11
03/10/89	600,000	11 11
03/13/89	1,814,000	11 19
03/14/89	642,000	п
03/15/89	556,000	n 11
03/16/89	556,000	11 11
03/17/89	532,000	IF IF
03/20/89	1,440,000	II II
03/21/89	536,000	и и
03/22/89	552,000	и и
03/23/89	460,000	п н
03/27/89	2,202,000	n n
03/28/89	458,000	H H
03/29/89	532,000	11 11
03/30/89	484,000	11 11
03/31/89	526,000	11 11
TOTAL VOLUME	16,336,000	No Discharge

Appendix

RADIATION STANDARDS FOR PROTECTION OF THE PUBLIC

Introduction

The primary standards for protection of the public from radiation are based on radiation dose. Radiation dose is a means of quantifying the biological damage or risk of ionizing radiation. The unit of radiation dose is the rem or the millirem (1 rem = 1,000 mrem). Radiation protection standards for the public are annual standards, based on the projected radiation dose from a year's exposure to or intake of radioactive materials.

Radiation dose is a calculated value. It is calculated by multiplying radioactivity concentrations in air and water or on contaminated surfaces by assumed intake rates (for internal exposures) or exposure times (for external exposure to penetrating radiation), then by the appropriate radiation dose conversion factors. That is

RADIATION DOSE =
(RADIOACTIVITY CONCENTRATION) X
(INTAKE RATE/EXPOSURE TIME) X
(DOSE CONVERSION FACTOR)

The radioactivity concentrations can be determined either by measurements in the environment or by calculations using computer models. These computer models perform airboine dispersion/dose modeling of measured

building radioactivity effluents and estimated diffuse source term emissions (e g , from resuspension from contaminated soil areas)

The assumed intake rates and dose conversion factors used are based on recommendations of national and international radiation protection advisory organizations, such as the National Council of Radiation Protection and Measurements (NCRP) and the International Commission on Radiological Protection (ICRP)

The radioactive materials of importance in calculating radiation dose to the public from Rocky Flats Plant activities include plutonium, uranium, americium, and tritium. The alpha radiation emissions from the plutonium, uranium, and americium are the primary contributors to the projected radiation dose.

Potential public radiation dose commitments, which could have resulted from Plant operations and from background (i.e., non-Plant) contributions, are calculated from average radionuclide concentrations measured at the Department of Energy (DOE) property boundary and in surrounding communities. Inhalation and water ingestion are the principal potential pathways of human exposure

Calculation of Potential Plant Contribution to Public Radiation Dose

Pending final revision of its DOE Order for adiation protection standards for the public, DOE adopted an interim radiation protection standard for DOE environmental activities to be implemented in CY1985 (Va85). This interim standard incorporates guidance from the National Council on Radiation Protection and Measurements (NCRP), as well as the Environmental Protection Agency Clean Air Act air emission standards (as implemented in 40 CFR 61, Subpart H). Included in the interim standard is a revision of the dose

limits for members of the public. Tables of radiation dose conversion factors currently used for calculating dose from intakes of radioactive materials were issued in July 1988 (US88a, US88b). The dose factors are based on the International Commission on Radiological Protection (ICRP) Publications 30 and 48 methodology and biological models for radiation dosimetry. The DOE interim standard and the dose conversion factor tables are used for assessment of any potential Rocky Flats. Plant contribution to public radiation dose. The DOE radiation standards for protection of the public are given below.

DOE RADIATION PROTECTION STANDARDS FOR THE PUBLIC

ICRP-, NCRP- RECOMMENDED STANDARDS FOR ALL PATHWAYS

OCCASIONAL EXPOSURES -

500 mrem/year

EFFECTIVE DOSE EQUIVALENT*

PROLONGED EXPOSURES -

(>5 YEARS)

100 mrem/year

EFFECTIVE DOSE EQUIVALENT

INDIVIDUAL ORGAN -

5,000 mrem/year

DOSE EQUIVALENT

EPA CLEAN AIR ACT STANDARDS FOR THE AIR PATHWAY ONLY

WHOLE BODY -

25 mrem/year

DOSE EQUIVALENT

ANY ORGAN -

75 mrem/year

DOSE EQUIVALENT

Secondary radioactivity concentration guides can be calculated from the primary radiation dose standards and used as comparison values for measured radioactivity concentrations DOE provided guidance for calculating these concentration guides - called "Derived Concentration Guides" - in a 1985 memorandum to its facilities (St85) Derived Concentration Guides (DCGs) are the concentrations which would result in an effective dose equivalent of 100 mrem from one year's chronic exposure or intake In calculating air inhalation DCGs, DOE assumes that the exposed individual inhales 8,400 cubic meters of air at the calculated DCG during the year Ingestion DCGs assume a water intake of 730 liters at the calculated DCG for the vear The following table lists the air and water DCGs for the principal radionuclides of interest at the Rocky Flats Plant

To determine compliance with the EPA air emissions standards, measured airborne of fluent radioactivity emissions and estimated radioactivity resuspension from soil are entered into the EPA-approved atmospheric dispersion/dose calculation computer model, AIRDOS-EPA, for calculation of the maximum radiation dose that an individual in the public could receive from the air pathway only

For comparison with the annual radiation dose standards for protection of the public, the maximum annual effective dose equivalent that a member of the public could receive as a result of Rocky Flats Plant activities is typically less than 1 mrem, or less than 1 percent of the recommended annual standard for all pathways

DOE DERIVED CONCENTRATION GUIDES FOR RADIONUCLIDES OF INTEREST AT THE ROCKY FLATS PLANT

AIR INHALATION:

Radionuclide	DCG (pCi/m³)
Pu-239, -240	0 02

WATER INGESTION:

<u>Radionuclide</u>	DCG (pCi/I)
Pu-239, -240	30
Am-241	30
U-233, -234, -238	500
H-3	2,000,000

References

- US88a DOE/EH-0070, "External Dose-Rate Conversion Factors for Calculation of Dose to the Public," U S Dept of Energy, Asst Secretary for Environment, Safety and Health, Office of Environmental Guidance and Compliance, July 1988
- US88b DOE/EH-0071, "Internal Dose Conversion Factors for Calculation of Dose to the Public," U S Dept of Energy, Asst Secretary for Environment, Safety and Health, July 1988
- Vass Vaughan, W. A., Asst Secretary, "Radiation Standards for Protection of the Public in the Vicinity of DOE Facilities," DOE memorandum from Environment, Safety and Health, August 5, 1985
- Stern, R. J., Director, "Preparation of Annual Site Environmental Reports for Calendar Year 1985," DOE memorandum, Office of Environmental Guidance, February 28, 1986
- *NOTE: "Dose equivalent" is a calculated value used to quantify radiation dose, it reflects the degree of biological effect from ionizing radiation. Differences in the biological effect of different types of ionizing radiation (e.g., alpha, beta, gamma, or x-rays) are accounted for in the calculation of dose equivalent.
- "Effective dose equivalent" is a calculated value used to allow comparisons of total health risk (based primarily on the risk of cancer

mortality) from exposures of different types of ionizing radiation to different body organs. It is calculated by first calculating the dose equivalent to those organs receiving significant exposures, multiplying each organ dose equivalent by a health risk weighting factor, and then summing those products. One millirem effective dose equivalent from natural background radiation would have the same health risk as one millirem effective dose equivalent from artificially-produced sources of radiation.